

## P2. Overview of PSA applications

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## **Outline of Presentation**



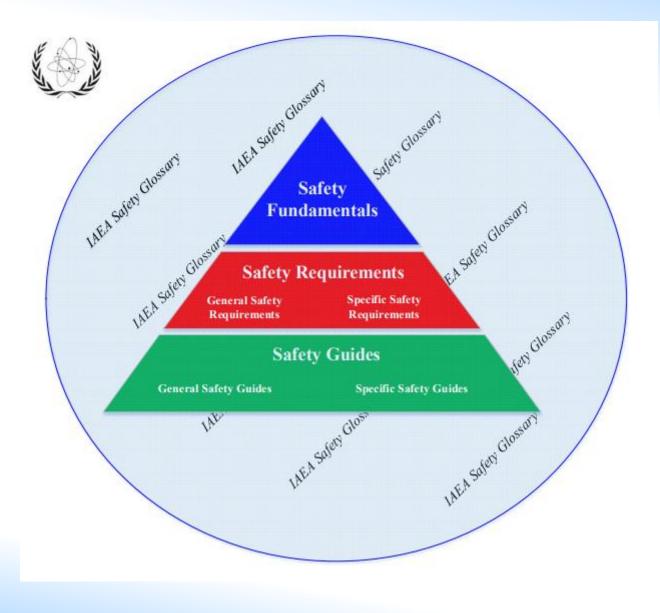
- PSA applications in IAEA publications
- Taxonomy of PSA applications
- Brief overview of PSA applications
- Quality of PSA models for applications

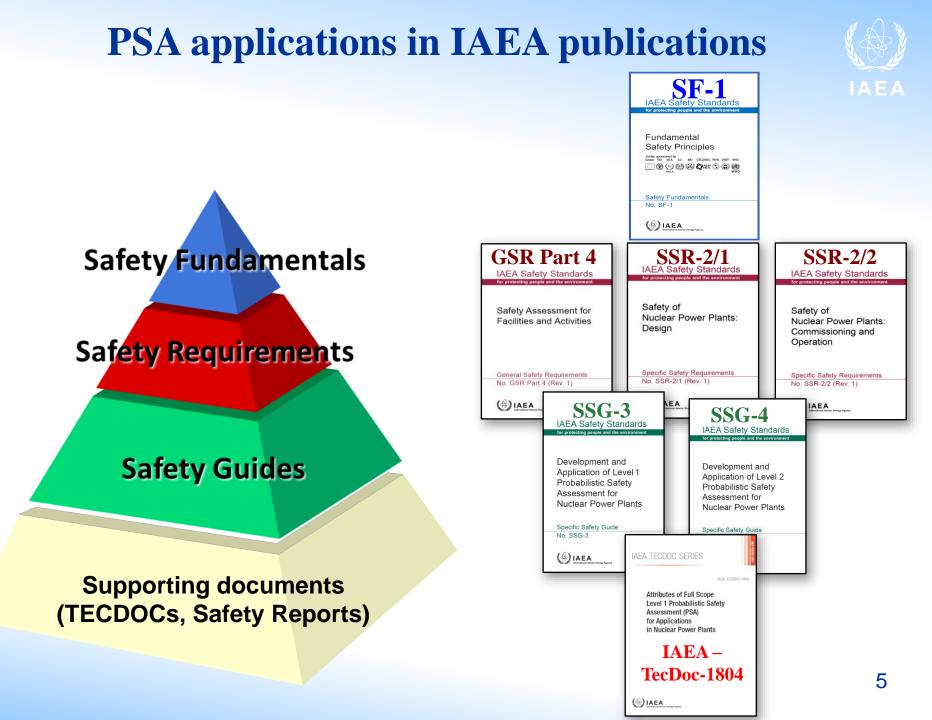


## **PSA applications in IAEA publications**

#### **Hierarchy of IAEA Safety Standards**







## **Relevant Statements from IAEA Safety Standards**



The fundamental safety objective is to protect people and the environment from harmful effects of ionizing radiation

**Principle 6:** Limitation of risks to individuals

*"Measures for controlling radiation risks must ensure that no individual bears an unacceptable risk of harm"* 

IAEA Safety Standards for protecting people and the environment
Fundamental Safety Principles Jointhy Spontored by Earth FAO WEA LO NO OECOMEA RHO UNEP WHO I I I I I I I I I I I I I I I I I I I
Safety Fundamentals No. SF-1

#### **Implications:**

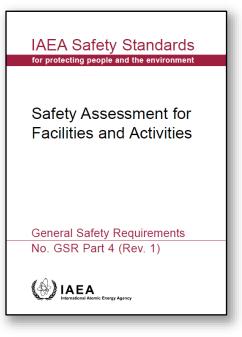
- 1) Risk associated with nuclear installations needs to be assessed
- 2) Guidance (criteria) for 'unacceptable risk' need to be established
- 3) Relevant measures (design features and procedures) provided

## PSA applications in IAEA publications: Safety Requirements on <u>Safety Assessment</u> (1/2)



- Safety assessment shall include a safety analysis, which consists of a set of different quantitative analyses for evaluating and assessing challenges to safety by means of **deterministic** and also **probabilistic** methods
- **Deterministic** and **probabilistic** approaches have to complement one another
- PSA to determine all significant contributing factors to the radiation risks arising from NPP
- PSA insights: system performance, reliability, interactions and weaknesses in the design, the application of defence-in-depth, and risks, that it may not be possible to derive from a deterministic analysis.

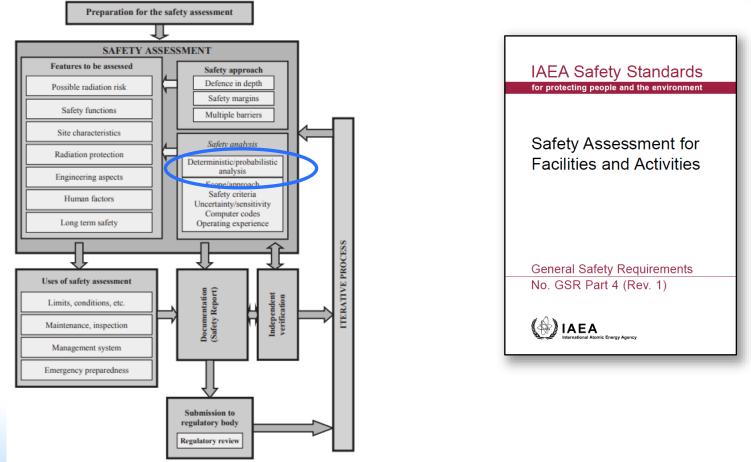
The results of the safety assessment shall be used to make decisions in an integrated, risk informed approach, by means of which the results and insights from the deterministic and probabilistic assessments and any other requirements are combined in making decisions on safety matters in relation to the facility or activity.



### PSA applications in IAEA publications: Safety Requirements on <u>Safety Assessment</u> (2/2)



#### **Overview of Safety Assessment Process**



## PSA applications in IAEA publications: Safety Requirements on <u>NPP Design</u>



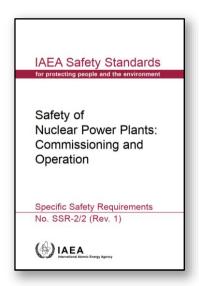
PSA shall be carried out throughout the design process to ensure that all safety requirements are met throughout all stages of NPP lifetime

- PSA shall be used in NPP design to:
  - Establishing balanced NPP design
  - Implement measures to avoid cliff-edge effect
  - Compare with risk criteria (probabilistic safety goals)
- PSA is used also for selection postulated IEs & DEC scenarios to be considered in NPP design (identification of input for deterministic analyses)
- PSA is used also for supporting NPP system, structures and components (SSCs) safety classification

IAEA Safety Standards						
for protecting people and the environment Safety of Nuclear Power Plants: Design						
Specific Safety Requirements						
No. SSR-2/1 (Rev. 1)						

## PSA applications in IAEA publications: Safety Requirements on <u>NPP Commissioning and Operation</u>

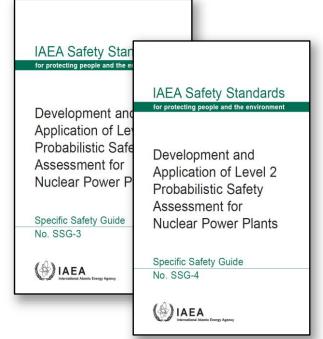
- To complement deterministic safety assessment (inter alia while conducting periodic safety review)
- PSA is to be used to determine frequency of maintenance, testing, surveillance and inspection of individual SSCs
- PSA shall be used, as appropriate, to demonstrate that the risks are not significantly increased due to maintenance activities
- Quality of PSA models shall be ensured for PSA applications



## PSA applications in IAEA publications: Safety Standards on Level-1&2 PSA



- Various PSA applications are presented in Section 10 of SSG-3 and Section 8 of SSG-4
- In PSA applications described in SSG-3 & SSG-4 insights from PSA should be used as part of Integrated Risk informed decision making (IRIDM) process considering:
  - Mandatory requirements
  - Insights from deterministic considerations (e.g. on Defence in Depth)
  - other applicable information (e.g. cost– benefit analysis, inspection findings, operating experience, doses to workers)

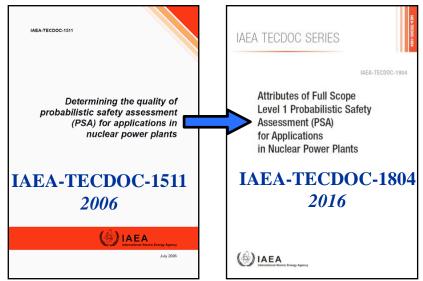


#### **IAEA-TECDOC-1804**



### Attributes to full-scope L1 PSA for Applications in IAE Nuclear Power Plants

- Provides an approach and detailed guidance for achieving the technical quality of PSA needed to support various PSA applications
- Covers a full-scope Level-1 PSA for NPPs
- Provides comprehensive list of PSA Applications with brief description of each PSA application
- 6 categories and about 40 specific applications



## **Taxonomy of PSA applications**



- <u>Safety assessment</u>: to assess the overall safety of the plant and to develop an understanding of the main contributing to risk (understanding risk profile)
- <u>Design stage</u>: to provide support for design improvements during design and pre-operational stage
- <u>NPP operation</u>: to provide support for plant daily operation (excluding permanent changes to design or operation practice)
- <u>Permanent changes to operating NPP</u>: to assess the safety significance of proposed permanent changes to the plant SSCs or operating procedures (help for Decision Making)
- Oversight activities: to support plant performance monitoring (both regulatory and industry)
- **Evaluation of safety issues**: risk significance



## **Brief description of PSA applications**



Application Category	Application Group	Specific Application			
1. SAFETY	1.1. SAFETY	1.1. Assessment of the overall plant safety			
ASSESSMENT	ASSESSMENT	1.2. Periodic safety review			
		1.3. Analysis of the degree of defence in depth and safety margin against beyond design basis site hazards, including correlated site hazards			
2. DESIGN STAGE	1.2. DESIGN STAGE	2.1. Application of PSA to support decisions made during the NPP design (plant under design)			
		2.2. Licensing of design			
		2.3. Optimization of protection against hazard events (e.g. fires, floods) and common cause failures, including consideration of correlated site hazards and hazard-induced fires and floods			
		2.4. Establishment of equipment reliability targets for manufacturers			
		2.5. Identification of R&D which are necessary to support the design			
		2.6. Development operator procedures and training programmes and support for Human Factors Engineering			



3. NPP OPERATION	3.1. NPP maintenance	3.1.1. Maintenance programme optimization						
		3.1.2. Risk informed house keeping						
		3.1.3. Risk informed support for plant ageing management programme						
		3.1.4. Risk informed on-line maintenance						
		3.1.5. Plant outage management						
	3.2. Accident mitigation and	3.2.1. Development and improvement of the emergency operating procedures						
	emergency planning	3.2.2. Support for NPP accident management (severe accident prevention, severe accident mitigation)						
		3.2.3. Support for NPP emergency planning						
	3.3. Personnel training	3.3.1. Improvement of operator training programme						
		3.3.2. Improvement of maintenance personnel training programme						
		3.3.3. Improvement of plant management training programme						
	3.4. Risk based configuration control/ Risk monitors	3.4.1. Configuration planning (e.g. support for plant maintenance and test activities)						
		3.4.2. Real time configuration assessment and control (response to emerging conditions)						
		3.4.3. Exemptions to TS and justification for continued operation						
		3.4.4. Dynamic risk informed TS						



4. PERMANENT CHANGES TO THE OPERATING PLANT	4.1. Plant changes	<ul> <li>4.1.1. NPP upgrades, backfitting activities and plant modifications</li> <li>4.1.2. Life time extension</li> </ul>				
	<ul> <li>4.2. Technical specification changes</li> <li>4.3. Establishment of graded QA programme for SSC</li> </ul>	<ul> <li>4.2.1. Determination and evaluation of changes to allowed outage time and changes to required TS actions</li> <li>4.2.2. Risk informed optimization of TS</li> </ul>				
		4.2.3. Determination and evaluation of changes to surveillance test intervals				
		4.2.4. Risk informed surveillance programme				
		4.2.5. Risk informed in-service inspections (RI-ISI)				
		4.3.1. Categorization of SSC for equipment risk significance evaluation				
		4.3.2. Evaluation of risk impact of changes to quality requirements				
	4.4. Risk informed special site protection measures	<ul> <li>4.4.1. Risk informed fire protection</li> <li>4.4.2. Risk informed internal flood protection</li> <li>4.4.3. Risk informed defence in depth for individual and correlated site hazards</li> </ul>				



5. OVERSIGHT ACTIVITIES	5.1. Performance monitoring	5.1.1. Planning and prioritization of inspection activities (regulatory and industry)				
		<ul><li>5.1.2. Long term risk based performance indicators</li><li>5.1.3. Short term risk based performance indicators</li></ul>				
	5.2. Performance	5.2.1. Assessment of inspection findings				
	assessment	5.2.2. Evaluation and rating of operational events				
6. EVALUATION OF SAFETY ISSUES	6.1. Risk evaluation	<ul> <li>6.1.1. Risk evaluation of corrective measures</li> <li>6.1.2. Risk evaluation to identify and rank safety issues</li> <li>6.1.3. Assessment of the safety importance of deviations between an existing plant design and updated/revise deterministic design rules or new information about the site hazards.</li> <li>6.1.4. Assessment of the significant of overall site risk for multiunit accidents</li> <li>6.1.5. Assessment of the significant of overall site risk from all radioactive sources</li> </ul>				
	6.2. Regulatory decisions	<ul><li>6.2.1. Long term regulatory decisions</li><li>6.2.2. Interim regulatory decisions</li></ul>				

## **Discussion on PSA applications**



- Further discussion on PSA applications is provided by the six categories and groups (where applicable) mentioned earlier
- The following aspects will be highlighted to characterize PSA applications:
  - <u>Main Contents</u> / goals of the application
  - Notes on the <u>Use of PSA</u> to support the application



## **Category 1. Safety assessment**

- 1) Assessment of the Overall Plant Safety
- 2) Periodical Safety Review

#### Main contents:

- Identification and ranking of important design and operational features
- Identification and ranking of dominant accident sequences, systems, components, human interactions and dependencies important for safety
- Comparison of the results against Safety goals

#### **Use of PSA:**

 Risk contributors and importance information is used to develop risk insights

## **Category 2. Design evaluation**



## **Risk-informed support for decisions made during NPP design**

#### Main contents:

- Identification of design weaknesses and effective areas for improvement in view of plant risk
- Investigation of design options
- Development of reliability targets for SSCs
- Risk significance of deviation from revised deterministic rules

- Additional assumptions are needed (lack of design details)
- Uncertainties are larger than for as-built plant

## Category 3.1. NPP Operation: NPP Maintenance



- 1) Maintenance program optimization
- 2) Risk informed support for plant ageing management program

#### **Main Contents:**

- Assessment, optimization and establishment of maintenance plans and procedures in view of plant risk
- Modelling of aging effects in PSA and identification of the ageing vulnerabilities among SSCs

- PSA model should be capable to include aging phenomena and component lifetime considerations
- PSA results can be used to
  - Optimize maintenance and ageing management programs 22

**Category 3.2. NPP Operation:** Accident Mitigation and Emergency Planning



- 1) Development and improvement of EOPs and SAMGs
- 2) Support for accident management and emergency planning
- Main Contents:
- Systematic assessment of plant vulnerabilities used to establish the EOPs, SAMGs and emergency planning

- L2 PSA is required to address severe accident mitigation strategies,
- Prioritize procedural changes
- Understand accident progression and success strategies

## Category 3.3. NPP Operation: Personnel Training



- 1) Improvement of operator training program
- 2) Improvement of maintenance personnel training program
- 3) Improvement of plant management training program

#### Main Contents:

- Improvement of operator and maintenance staff training programs based on insights and information from the PSA
- Providing plant management with the integral understanding of the techniques, applications and implications of PSA

- L1 PSA covering all operating and shutdown modes and both internal and external plant hazards is required
- PSA results can be used to focus on potential risk significant accident sequences, operator actions, maintenance activities

# Category 3.4 NPP Operation:

- 1) Configuration planning (e.g., support for plant maintenance/tests)
- 2) Real time configuration assessment and control
- **3) Exemptions to TS and justification for continued operation**

#### Main Contents:

• The reduction of risk peaks and the control of the cumulative or average risk (e.g., Risk Monitor)

- Can provide useful support to TS exemption justifications
  - The purpose of TS framework is to keep plant features within the licensing basis

## Category 4.1. Permanent Changes to the Operating Plant: *Plant Changes*



- 1) NPP upgrades and plant modifications
- 2) Life time extension

#### Main Contents:

- Identification of weaknesses and effective areas for improvement in plants design and operational features
- Providing additional information for regulators while licensing the lifetime extension

- Change in CDF used to justify acceptable risk impacts and to determine risk significance
- Involves modelling of aging effects in PSA

# Category 4.2. Permanent changes to the operating plant: *TechSpec Changes*



- **1) Determination and evaluation of changes to AOTs**
- 2) Risk informed optimisation of TS
- 3) Determination and evaluation of changes to STI
- 4) Risk informed in-service testing (IST)
- 5) Risk informed in-service inspections (ISI)

#### Main Contents:

- Optimization of the STIs vs impact on equipment reliability
- Re-evaluation of required actions in TS based on risk
- Optimization of AOT period for each SSC
- Support the IST programme vs relative risk significance
- Ranking the elements for inspection

#### **Use of PSA**:

• To estimate the risk significance of the changes in TS

# Category 4.3 Permanent changes to the operating plant: Graded QA

- 1) Equipment risk significance evaluation
- 2) Evaluation of risk impact of changes to QA requirements

#### Main Contents:

• Determination of the relative safety significance of plant equipment

- Risk importance measures of affected SSCs (e.g. F-V, RAW) are used to classify the risk and safety significance of SSCs
- Change in risk metrics (△CDF, △LERF) are used to determine the risk significance and risk acceptability of the proposed change

## Category 5.1. Oversight Activities: Performance Monitoring



- 1) Planning and prioritization of inspection activities
- 2) Long-term risk-based performance indicators
- 3) Short-term risk based performance indicators

#### Main Contents:

- Regulatory and industry inspections on risk significant issues and equipment
- Monitoring plant behaviour in order to update the calculated average CDF

- PSA provides information on changes in CDF due to plant events and risk associated with planned activities
- The PSA results can be used to determine the appropriate set of performance indicators

## Category 5.2. Oversight Activities: Performance Assessment



- 1) Assessment of inspection findings
- 2) Evaluation and rating of operational events

#### **Main Contents:** Self-explanatory

- Estimation of risk-significance associated with inspection findings
- PSA based extrapolation of operational events to accident scenarios with serious consequences
  - Can provide an estimate, in terms of a conditional probability, of the available margin for an accident with unacceptable consequences
  - The PSA model must be capable of evaluating the appropriate impacts assessed for the event

## Category 6.1 Evaluation of Safety Issues: *Risk Evaluation*



- **1)** Risk evaluation to identify and rank safety issues
- 2) Risk evaluation of corrective measures

#### Main Contents:

- Identification or evaluation of risk-significance of the plant specific and generic safety issues
- Determination of the risk-effective corrective measures

- Based on PSA insights corrective measures regarding safety issues are developed
  - Change in risk metrics (△CDF, △LERF) are used to determine the risk significance and risk acceptability of the proposed change based on risk characterization
- PSA is used for evaluating the relative importance of existing and new safety issues
  - Contributors to risk and risk importance measures are used to identify and rank safety issues

## Category 6.2 Evaluation of Safety Issues: Regulatory Decisions



- 1) Long-term regulatory decisions
- 2) Interim regulatory decisions

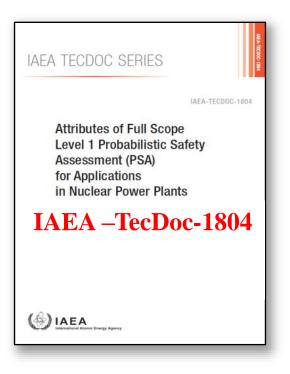
Main Contents:

Maintaining or reducing risk level

- to guide long term prioritization of regulatory objectives and requirements, and of related safety research
- to alleviate a regulatory concern, while longer term solutions can be evaluated
- Issues that typically require an interim decision
  - Need for regulatory action in response to an event
  - One-time exemptions from TS
  - Temporary modifications to hardware or procedures



## **Quality of PSA models for PSA applications**

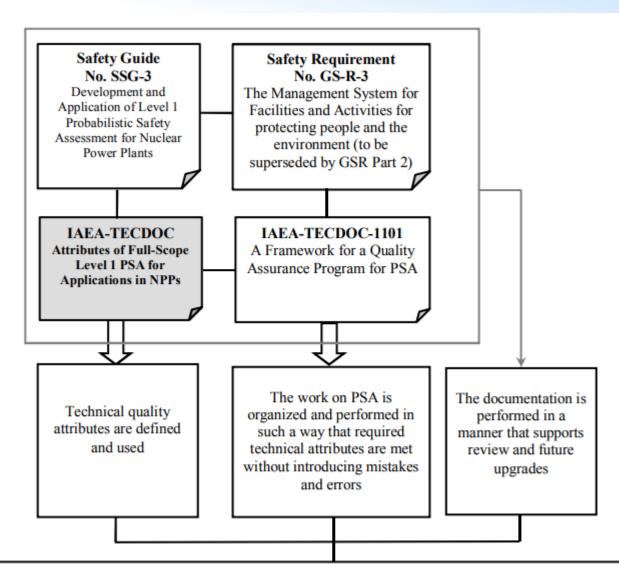


## **PSA quality** ↔ **PSA applications**



- PSA applications and RIDM require that PSAs used to support those applications have certain characteristics:
  - Scope;
  - Degree of detail;
  - Technical adequacy of the modelling;
  - Capability and flexibility,
  - Quality and type of the data used,
  - Assumptions made in modelling important aspects
- Features of a PSA that are necessary to support specific applications vary with the application
- Need for a basis for judging the technical quality of the PSA used to support an application

## Framework of PSA technical quality



#### Provision of Technical Quality of PSA Results for Applications

## **PSA elements and associated abbreviations**

1.	Plant Operational States Analysis	OS
2.	Hazards Events Analysis	HE
3.	Initiating Events Analysis	IE
4.	Accident Sequence Analysis	AS
5.	Success Criteria Formulation and Supporting Analysis	SC
6.	Systems Analysis	SY
7.	Human Reliability Analysis	HR
8.	Data Analysis	DA
9.	Dependent Failures Analysis	DF
10.	Model Integration and Risk Metric Frequency Quantification	MQ
11. 12	Results Analysis and Interpretation Maintenance and Upgrade of the PSA	RI MU

## **Quality of Level 1 PSA: Attributes**



#### TABLE 7.2-B ATTRIBUTES FOR AS ANALYSIS: TASK AS-B 'DEFINITION OF SUCCESS AND NON-SUCCESS END STATES AND KEY SAFETY FUNCTIONS'

Task / GA		of Task/General Attributes escription of Special Attributes (in Italics)	Rationale/Comments/Examples for: General Attributes and Special Attributes (in Italics)				
AS-B	external hazards necessary to read	ng event group for internal events, internal hazards, and for each POS the key safety functions that are th a success end state are identified. Success and non- es are clearly defined.	COMMENT: If the scope of the PSA does not include certain hazards or LPSD POS, then these do not have to be considered.				
AS-B01	provides the poss the success end s available tools (c etc.).	non-success states are defined in a manner that sibility to justify the achievement (non-achievement) of state for each accident sequence with the use of e.g. thermal hydraulic analysis, tests and experiments, e identified as success or non-success; no end state is	RATIONALE: Undefined end states prevent the capability for a useful interpretation of the results.				
	AS-B01-S1 For accident sequences resulting from initiating events affecting multiple reactor units, the end states must account for the success and non-success end state status of all affected units.		COMMENT: An initiating event that impacts a combination of N reactors on the site may result in any possible combination of reactor end states ranging from all N reactors with successful end states to all N reactors with unsuccessful end states.				
AS-B02	Systems and pro safety functions availability of sp (e.g. information controlled equip For each safety f	ng event group the key safety functions are identified. cedurally directed operator actions required to perform are identified for each IE group with account for ecific equipment and conditions for operator actions available for operator, acceptability for manually ment, time window, etc.). function, system models are developed with account for defined for specific IE group and AS.					
AS-B03		or the achievement of stable success end state conditions ach AS with account of all uncertainties associated with ols.					

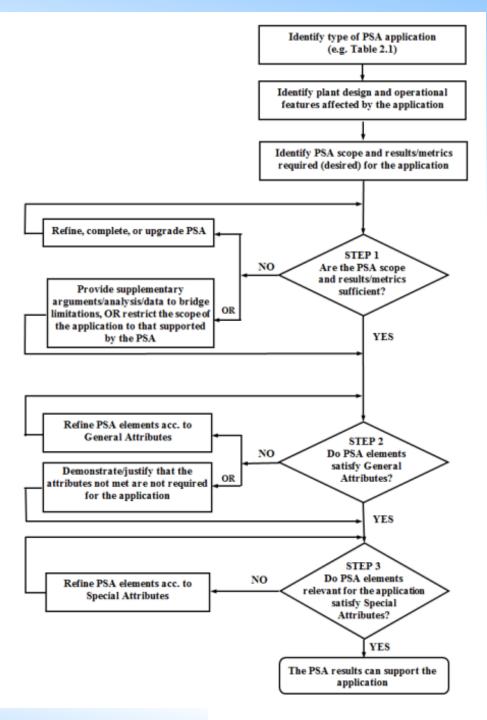
## **Quality of Level 1 PSA: Attributes**



## Mapping the special attributes of PSA elements to PSA applications (those attributes should be assured before using the PSA model for particular application)

PSA Application Group/	PSA Elements								
PSA Application	IE	AS	SC	SY	HR	DA	DF	MQ	Other
3.1.3 Risk informed support for plant ageing management programme	正-H02-S1	-	-	<u>SY-B19-S1</u> <u>SY-B22-S1</u>	-	<u>DA-E01-S1</u>	DF-F01-S1 DF-G01-S1	-	-
3.1.4 Risk informed on-line maintenance	-	-	-	-	-	-	-	MQ-A01-S1 MQ-C02-S1	-
3.1.5 Plant outage management	-	-	SC-A03-S1	-	HR-G02-S1 HR-G04-S1 HR-K02-S1 HR-K05-S1	-	-	-	OS-A03-S1 OS-C01-S2
3.2 Accident mitigation and en	nergency planı	ning							
3.2.1 Development and improvement of the emergency operating procedures	-	AS-B03-S1 AS-C03-S1 AS-C04-S1 AS-C16-S1 AS-C05-S1 AS-C06-S1 AS-C08-S1	-	-	HR-G02-S1 HR-G04-S1	-	DF-F01-S1 DF-G01-S1	-	-
3.2.2 Support for NPP accident management (severe accident prevention, severe accident mitigation)	Œ-B01-S1	-	-	-	<u>HR-G02-81</u>	DA-D06-81	DF-F01-S1 DF-G01-S1	-	OS-A01-S1 HE-D04-S1
3.2.3 Support for NPP emergency planning	IE-B01-S1	AS-C05-S1	-	-	-	-	-	-	- 38

General procedure for determination of technical quality of PSA for applications



## **Quality: PSA review**



- It is a widely accepted practice for the organization conducting a PSA to commission an independent peer review of the PSA from an outside body, sometimes from a different State, to provide a degree of assurance that the scope, modelling and data are adequate
- The experts involved in the review of the PSA should not be engaged in any activities relating to performance of the PSA under consideration and should represent an organization that is independent of the developer of the PSA

## **PSA applications: summary**



- Enhancement of safety decision making by complementing the traditional deterministic approach with a systematic probabilistic approach that is consistent and predictable
- Efficient use of regulatory resources and reduction in unnecessary burden on licensees
- Continued focus on aspects that contribute to risks of NPP operation
- Requires maintaining updated risk analyses models, tools, and understanding of risk implications of plant design features and processes incorporating operating experiences





**Questions?**