# Operational Safety with OEF



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**Korea Institute of Nuclear Safety** 

#### CONTENTS

- Introduction
- II. OEF Requirements for NPPs
- III. Reporting, Investigation and Evaluation of NPP Events in Korea
- IV. The National OEF Program in Korea

## Introduction



#### Safety of Nuclear Power Plants (NPPs)

#### Proper **Proper** - Site Selection - Management - Design + Evaluation - Operation - Construction - Maintenance - Commissioning + Proper transition to decommissioning

# < Contents of IAEA SSR-2/2 Safety of Nuclear Power Plants : Commissioning and Operation >

#### CONTENTS

| 1. | INTRODUCTION  | 1   | 5. | OPERATIONAL SAFETY PROGRAMMES   | 2 |
|----|---|-----|----|---|---|
|    | Background (1.1–1.4).   | 1   |    | Requirement 17: Consideration of objectives of nuclear security         |   |
|    | Objective (1.5)   | 1   |    | in safety programmes (5.1).   | 2 |
|    | Scope (1.6)   |     |    | Requirement 18: Emergency preparedness (5.2–5.7)                        | 2 |
|    | Structure (1.7–1.8)   | 2   |    | Requirement 19: Accident management programme (5.8–5.9)                 | 2 |
|    |   |     |    | Requirement 20: Radiation protection (5.10–5.16)                        | 2 |
| )  | SAFETY OBJECTIVE AND PRINCIPLES (2.1–2.2)                     | 2   |    | Requirement 21: Management of radioactive waste (5.17–5.20)             | 2 |
|    |   | -   |    | Requirement 22: Fire safety (5.21–5.25)                                 | 2 |
| 3  | THE MANAGEMENT AND ORGANIZATIONAL                             |     |    | Requirement 23: Non-radiation-related safety (5.26)                     | 2 |
|    | STRUCTURE OF THE OPERATING ORGANIZATION                       | 4   |    | Requirement 24: Feedback of operating experience (5.27–5.33)            | 2 |
|    | STRUCTURE OF THE OFERTIEN ONORMER THON                        |     |    |   |   |
|    | Requirement 1: Responsibilities of the operating              |     | 6. | PLANT COMMISSIONING   | 2 |
|    | organization (3.1–3.3)  | 4   |    | D :   |   |
|    | Requirement 2: Management system (3.4–3.7)                    | 6   |    | Requirement 25: Commissioning programme (6.1–6.15)                      | 2 |
|    | Requirement 3: Structure and functions of the                 | · · | -  | DI ANTE OPERATIONA  |   |
|    | operating organization (3.8–3.9)                              | 7   | 7. | PLANT OPERATIONS  | 3 |
|    | Requirement 4: Staffing of the operating                      | ,   |    | P : .060 : 4 (71.70   |   |
|    | organization (3.10–3.13)                                      | 8   |    | Requirement 26: Operating procedures (7.1–7.6)                          | 3 |
|    |   | 0   |    | Requirement 27: Operation control rooms and control equipment (7.7–7.9) | 3 |
| 4. | MANAGEMENT OF OPERATIONAL SAFETY                              | 9   |    | Requirement 28: Material conditions and<br>housekeeping (7.10–7.12)     | 2 |
|    | Requirement 5: Safety policy (4.1–4.5).                       | 9   |    | Requirement 29: Chemistry programme (7.13–7.17)                         |   |
|    | Requirement 6: Operational limits and conditions (4.6–4.15)   | 10  |    | Requirement 30: Core management and fuel handling (7.18–7.29)           |   |
|    | Requirement 7: Qualification and training of                  | 10  |    | Requirement 50. Core management and ruer nandring (7.16–7.29)           | 2 |
|    | personnel (4.16–4.24).  | 11  | 8. | MAINTENANCE, TESTING, SURVEILLANCE AND                                  |   |
|    | Requirement 8: Performance of safety related                  | 11  | 0. | INSPECTION  | 3 |
|    | activities (4.25–4.32)  | 13  |    | INST De Hor   | _ |
|    | Requirement 9: Monitoring and review of safety                | 15  |    | Requirement 31: Maintenance, testing, surveillance and                  |   |
|    | performance (4.33–4.37)                                       | 14  |    | inspection programmes (8.1–8.17).                                       | 3 |
|    | Requirement 10: Control of plant configuration (4.38)         | 15  |    | Requirement 32: Outage management (8.18–8.24)                           |   |
|    | Requirement 11: Management of modifications (4.39–4.43)       | 16  |    | requiencia 32. Guage management (6.26 6.27)                             | _ |
|    | Requirement 12: Periodic safety review (4.44–4.47).           | 17  | 9. | PREPARATION FOR DECOMMISSIONING   | 3 |
|    | Requirement 13: Equipment qualification (4.48–4.49)           | 17  |    | The first of the commission with  | _ |
|    | Requirement 14: Ageing management (4.50–4.51)                 | 18  |    | Requirement 33: Preparation for decommissioning (9.1–9.6)               | 3 |
|    |   |     |    |   | - |
|    | Requirement 15: Records and reports (4.52)                    | 18  | RE | FERENCES  | 4 |
|    | Requirement 16: Programme for long term operation (4.53–4.54) | 19  |    | NTRIBUTORS TO DRAFTING AND REVIEW DIES FOR THE ENDORSEMENT OF           | 4 |
|    |   |     |    | AEA SAFETY STANDARDS  | 4 |

# II. OEF Requirements for NPPs



## II.1 IAEA requirements

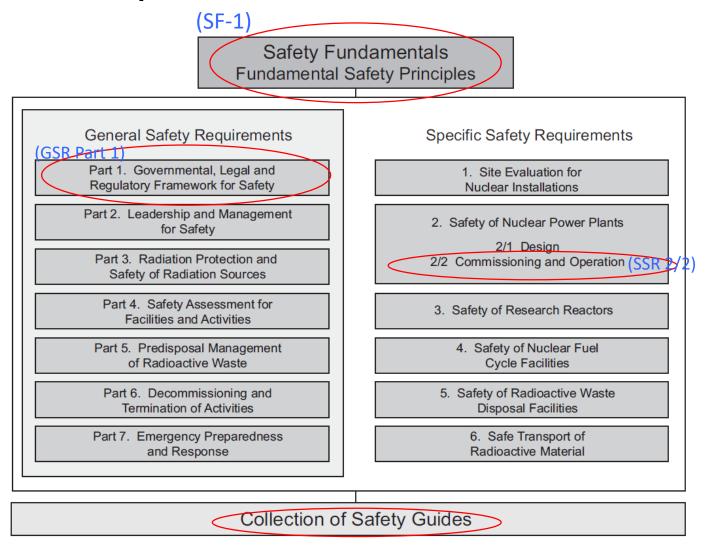


FIG. 1. The long term structure of the IAEA Safety Standards Series.





7A€A - INFCIRC/449 -

GENERAL Distr.
Original: ARABIC, CHINESE,
ENGLISH, FRENCH, RUSSIAN,
SPANISH

#### CONVENTION ON NUCLEAR SAFETY

# 1. IAEA-INFCIRC/449 Convention on Nuclear Safety (1994.6.17)

- Article 19. Operation

- 1. The Convention on Nuclear Safety was adopted on 17 June 1994 by a Diplomatic Conference convened by the International Atomic Energy Agency at its Headquarters from 14 to 17 June 1994. The Convention will be opened for signature on 20 September 1994 during the thirty-eighth regular session of the Agency's General Conference and will enter into force on the ninetieth day after the date of deposit with the Depositary (the Agency's Director General) of the twenty-second instrument of ratification, acceptance or approval, including the instruments of seventeen States, having each at least one nuclear installation which has achieved criticality in a reactor core.
- The text of the Convention as adopted is reproduced in the Annex hereto for the information of all Member States.

Each Contracting Party shall take the appropriate steps to ensure that:

(vii) programmes to collect and analyse operating experience are established, the results obtained and the conclusions drawn are acted upon and that existing mechanisms are used to share important experience with international bodies and with other operating organizations and regulatory bodies;

#### 2. SF-1 Fundamental Safety Principles

Principle 3 : Leadership and management for safety

#### Fundamental Safety Principles

Jointly sponsored by Euratom RAO IAEA ILO IMO OECDINEA PAHO UNEP WHO

Safety Fundamentals
No. SF-1

3.17 Despite all measures taken, accidents may occur. The precursors to accidents have to be identified and analysed, and measures have to be taken to prevent the recurrence of accidents. The <u>feedback of operating experience</u> from facilities and activities — and, where relevant, from elsewhere — <u>is a key means of enhancing safety</u>. Processes must be put in place for the <u>feedback and analysis</u> of operating experience, including initiating events, accident precursors, near misses, accidents and unauthorized acts, so that lessons may be learned, shared and acted upon.

#### 3. GSR Part 1 Governmental, Legal and Regulatory Framework for Safety

- Requirement 15 : Sharing of operating experience and regulatory experience

The regulatory body shall make <u>arrangements for analysis</u> to be carried out to identify lessons to be learned from operating experience and regulatory experience, including experience in other States, and for the dissemination of the lessons learned and for their use by authorized parties, the regulatory body and other relevant authorities.

Related items: 3.3~3.5

IAEA Safety Standards for protecting people and the environment

Governmental, Legal and Regulatory Framework for Safety

General Safety Requirements Part 1 No. GSR Part 1



# 4. SSR-2/2 Safety of Nuclear Power Plants : Commissioning and Operation

- Requirement 24: Feedback of operating experience

The operating organization shall establish an operating

experience programme to learn from events at the plant and events in the nuclear industry and other industries worldwide.

Related items: 5.27~5.33

Safety of Nuclear Power Plants: Commissioning and Operation

IAEA Safety Standards

for protecting people and the environment

Specific Safety Requirements
No. SSR-2/2



#### 5. SSG NS-G-2.10 PSR of NPPs (2003)

Use of experience from other plants and research findings

Objective (4.39): The objective of the review of experience from other plants and research findings is to determine whether

there is <u>adequate feedback of</u> safety experience from other nuclear power plants and of the findings of research.

SAFETY
STANDARDS
SERIES

Description (4.40)

Periodic Safety Review of Nuclear Power Plants



No. NS-G-2.10



#### 6. SSG No. SSG-25 PSR for NPPs (2013)

- Safety factor 9: Use of experience from other plants and research findings
- 5.103. Experience from other nuclear power plants, and sometimes from non-nuclear facilities, together with research findings, can reveal previously unknown safety weaknesses or can help in solving existing problems. Reference [2] requires the operating organization to obtain and evaluate information on operating experience at other plants and to derive lessons for its own operations. ...

Objective (5.104), Scope and Tasks (5.105~106),

Methodology (5.107~5.110)



#### 6. SSG No. SSG-50 OEF for Nuclear Installations

This publication has been superseded by SSG-50

#### IAEA Safety Standards

for protecting people and the environment

A System for the Feedback of Experience from Events in Nuclear Installations



#### IAEA Safety Standards

for protecting people and the environment

Operating Experience Feedback for Nuclear Installations

Safety Guide No. NS-G-2.11



Specific Safety Guide

No. SSG-50





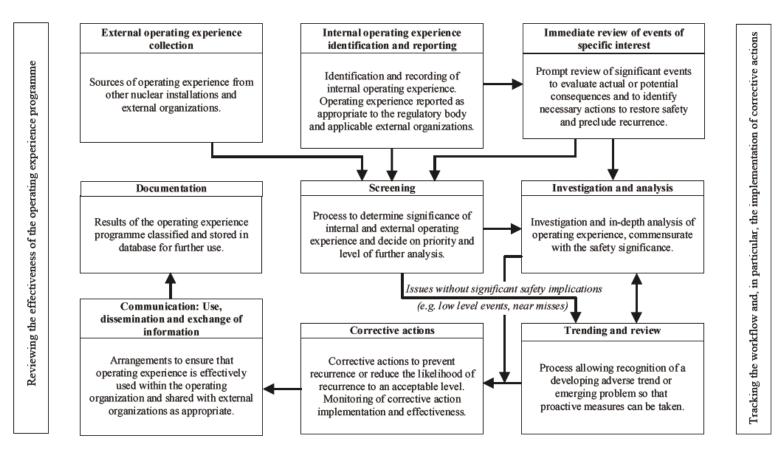


FIG. 1. Typical operating experience programme.

#### Structure of IAEA OEF requirements

- IAEA-INFCIRC/449 Convention on Nuclear Safety (Article 19,(vii))
   : OEF Programmes
  - SF-1 Fundamental Safety Principles (Principle 3, 3.17)
    - : OEF is a key means of enhancing safety.
    - General Safety Requirements Part 1 (Requirement 15, 3.3~3.5)
      - : Sharing of OE and Regulatory experience
    - Specific Safety Requirements 2/2 (Requirement 24, 5.27~5.33)
      - : Feedback of OE
  - Safety guides
    - NS-G-2.10 (PSR, 2003) (Safety factors, 4.39~4.40)
      - : Use of experience from other plants and research findings
    - SSG-25 (PSR, 2013) (Safety factors, 5.103~5.110)
      - : Use of experience from other plants and research findings
    - SSG-50
      - : Operating Experience Feedback for Nuclear Installations

#### II.2 OEF requirements in Korea

- 1. Nuclear Safety Act
  - Article 11(Standards for construction permit)
    - 1. <u>Technical capability</u> necessary for construction of a nuclear power reactor and related facilities...

. . .

- Article 21(Standards for operating license)
  - 1. <u>Technical capability</u> necessary for the operation of the nuclear power reactor and related facilities...

. . .

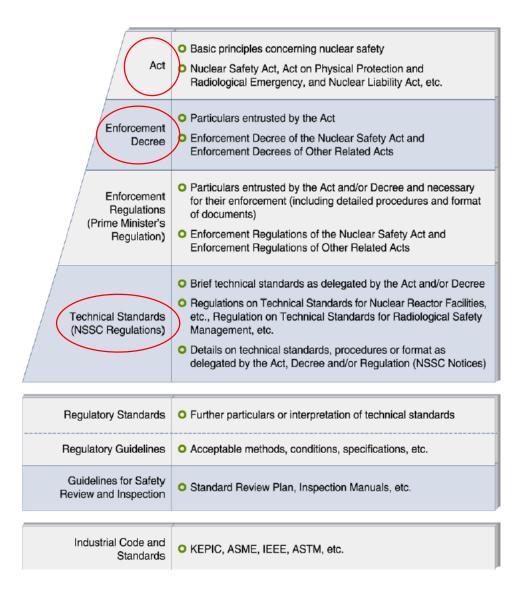


Fig. Legal Framework for Nuclear Safety Regulation in Korea



#### 2. Enforcement regulations for the Nuclear Safety Act

- Article 8 (Technical capability necessary for construction)
  - 1. Organization for the construction ...
  - 2. Engineering and technical support organization ...
  - 3. Qualification and experience of the personnel...
  - 4. ... a system whereby the construction cases of reactor facilities can be analyzed for reflection in design and construction ...
  - 5. Test and inspection plan for the SSCs ...

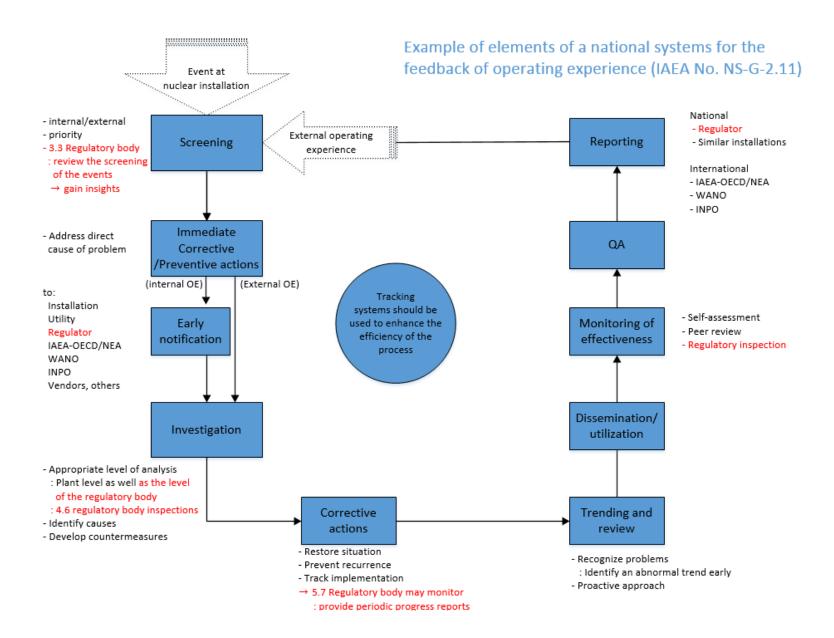
- 3. Regulations on Technical Standards for Nuclear Reactor Facilities, Etc.
  - Article 58(Operational Experience Feedback)
    - The operator of a nuclear power reactor shall take each of the following <u>actions in order to reflect operating experience</u> <u>systematically</u>
    - 1. OE shall be collected, analyzed, and managed...
    - 2. ... shall be reflected in plant facilities, safety related criteria, procedures and training program.
- 4. NSSC Notice 2020-03, Regulation of Reporting and Public Announcement of Events for the Nuclear Power Utilization Facilities

#### Regulatory requirements on OEF in Korea

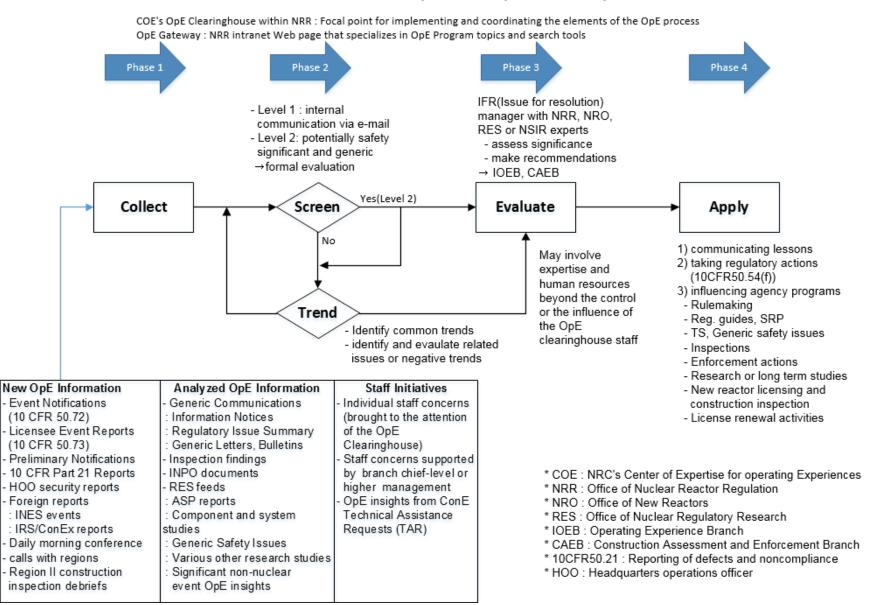
- Nuclear Safety Act (Article 11, 21)
  - : Technical capability for construction/operation
- One of the technical capability for construction
  - Enforcement regulations for the Nuclear Safety Act (Article 8)
    - : Reflection on the previous construction cases
- One of the technical capability for operation
  - Regulations on Technical Standards for Nuclear Facilities, Etc.(Article 58)
    - : OEF during operation

# III. Reporting, Investigation and Evaluation of NPP Events in Korea





#### Overview of NRC Reactor OpE Process(NRR-LIC-401)





## **III.1 Event Reporting Criteria**

#### Introduction

- Legal basis of the event reporting
  - Notice of the NSSC No. 2018-03 (Regulation on reporting and public announcement of accidents and incidents for nuclear power utilization facilities)
- Contents of the notice
  - Items to report
  - Methods and Procedures of Reporting
  - Rating of the events
  - Release of the event information
  - Investigation and utilization of the results

#### A. Events applicable to all nuclear facilities

|   | Report Deadline  |                 | Dating of        |
|---|------------------|-----------------|------------------|
| Events to Report  | Verbal<br>report | Detailed report | Rating of Events |
| 1. Incidents related to the transportation or packaging of radioactive materials, (leakage, theft or exposure,) | <30 min          | 60 days         | 0                |
| Theft, loss, fire or other accidents of radiation generating devices or radioactive materials                   | <30 min          | 60 days         | 0                |
| 3. Declaration of radiation emergency   | -                | -               | $\circ$          |
| Surface contamination at areas other than controlled radiation area   | <30 min          | 60 days         | 0                |
| 5. Abnormal increase of radiation level   | <30 min          | 60 days         |                  |
| 6. Release of radioactive material into environment   | <30 min          | 60 days         |                  |
| 7. Occurrence of human casualties   | <30 min          | -               | -                |

#### A. Events applicable to all nuclear facilities (continued)

|  | Report Deadline  |                 | Deting of        |
|--|------------------|-----------------|------------------|
| Events to Report   | Verbal<br>report | Detailed report | Rating of Events |
| 8. Abnormal radiation exposure   | <30 min          | 60 days         |                  |
| 9. Threat to the safe operation of the facility as a result of such events as fire or toxic gas in the facility. | <30 min          | 60 days         | -                |
| 10. Release of radioactive material which exceeds regulatory limits at the EAB                                   | <30 min          | 60 days         | 0                |
| 11. Loss of control for radioactive material   | <30 min          | 60 days         | 0                |
| 12. In case emergency measures are taken due to natural disasters (fires, storm, tsunami, etc.)                  | 4 hours          | 60 days         | -                |
| 13. In case emergency measures are taken due to industrial disasters   | 4 hours          | 60 days         | -                |

#### B. Events applicable to power and research reactor facilities

|  | Report D         | eadline         | Deting of        |
|--|------------------|-----------------|------------------|
| Events to Report   | Verbal<br>report | Detailed report | Rating of Events |
| 1. Excess of safety limits/setpoints for safety systems  | <30 min          | 60 days         | 0                |
| 2. Violations of the actions required for the LCO in TS  | 4 hours          | 60 days         |                  |
| 3. Failure of the safety system actuation(RPS,ECCS,)   | 4 hours          | 60 days         | 0                |
| 4. Earthquake etc. with actuation of the monitors  | <30 min          | 60 days         | -                |
| 5. Failure of the primary parameter displaying group in MCR, or functional loss of the notification system | 4 hours          | 60 days         | 0                |
| 6. Functional failure of the systems related to the safe storage of spent fuels.                           | 8 hours          | 60 days         | 0                |
| 7. Failure of safety related SCCs of the spent fuel storage cask or severe degradation of the cask.        | 8 hours          | 60 days         | 0                |

#### C. Events applicable to power reactor facilities

|  | Report Deadline  |                 | Deting of        |
|--|------------------|-----------------|------------------|
| Events to Report   | Verbal<br>report | Detailed report | Rating of Events |
| Automatic or manual reactor trips due to failures in the facilities or human errors              | 4 hours          | 60 days         | 0                |
| 2. Actuation of any of the Engineered Safety Features ( ESF) including ECCS, CIS, AFS, EDG, etc. | 4 hours          | 60 days         | 0                |
| 3. Reduction of power to satisfy the LCOs  | 4 hours          | -               | -                |
| 4. Unanticipated leakage of radioactive materials  | <30 min          | 60 days         | 0                |
| 5. leakage at the pressure boundary of RCS   | 8 hours          | 60 days         | 0                |
| 6. Loss of shutdown cooling system or residual heat removal system                               | 4 hours          | 60 days         | 0                |

#### Methods and Procedures of Reporting

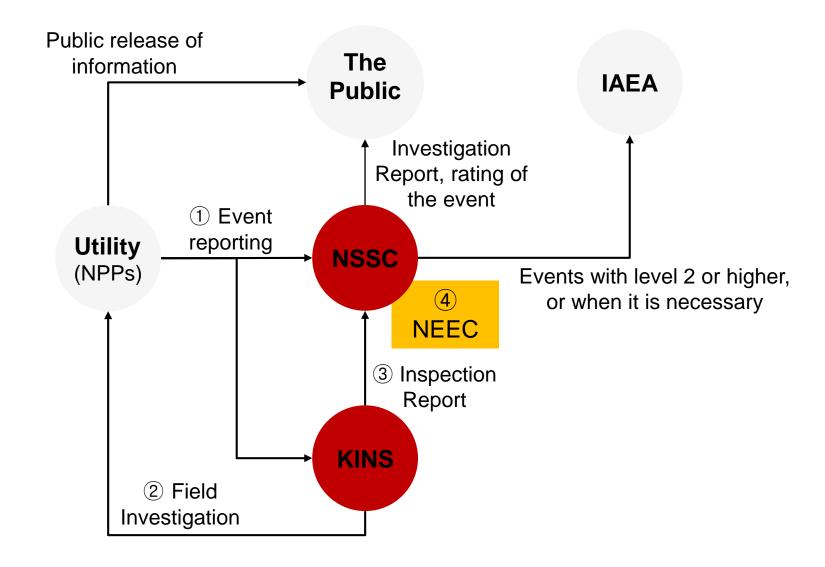
- Verbal and written reporting to NSSC
  - Verbal reporting: with available means of communication
    - → Written report should be submitted within next working day.
    - → The report should include the material released to the press.
  - Written report within the times specified in the Table.
- Rating of the events
  - Licensee shall make provisional rating on their initial written reporting.
  - Official rating by the Nuclear Event Evaluation Committee (NEEC)

#### Release of information to the public

- Information release to the public
  - Licensees shall post the information on the internet and release it to the press within 24 hours.
- Reporting to IAEA
  - NSSC shall send the results of rating and background information of the incidents for the following cases.
    - Provisional rating is level 2 or higher.
    - IAEA is interested in,
    - Dissemination of the information is necessary.

## **III.2 Event Investigation**

#### Response to the events in NPPs



#### Roles of the related organizations

- 1. Utility: report the events (notice of NSSC 2014-17)
- 2. KINS: dispatch a team to perform investigation
  - Causes and Effects
  - Operator's response
  - Response of the safety systems during the event
  - Corrective actions and measures to prevent recurrence
- 3. NSSC: approval of restart based on the investigation.
- 4. Tracking of the corrective actions
  - Utility reports the current status quarterly.
  - Periodic inspection by KINS

## Regulatory investigation

#### Objective

To Check whether safety functions were maintained properly.

#### Contents of the investigation

- Behavior of plant safety systems during the transient
  - Any challenges to the safety functions?
- Licensee's response activities
  - Mitigate of the adverse effects
  - investigation on the causes and corrective action items
- Confirm whether the safety functions were maintained properly and corrective actions were made correctly to prevent recurrence of similar events.

#### Response to events in KINS

#### KINS investigation team

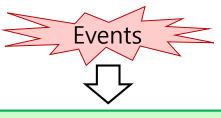
- Dept. of Reactor analysis
- Dept of SA and Risk Assessment
- Dept. of Mechanical & Material Engineering
- Dept. of Instrument, Control & Electrical System
- Dept. of Reactor System
- Dept. of Structural Systems & Site Safety Evaluation
- Dept. of Radiation Protection & Radioactive Waste Safety
- Dept. of Quality Assurance

+ Dept. of Operational Safety Analysis (Coordination, Operational aspects)









Incident report to NSSC, KINS



Progress report during investigation





Investigation Report to NSSC

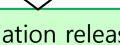


Tracking on the corrective actions (http://cats.kins.re.kr)





Nuclear event evaluation committee (Rating events, quarterly)



Information release (http://opis.kins.re.kr)

## Plant restart after a reactor trip

- Process
  - Reactor shutdown → KINS field inspection → Report the result
  - → NSSC : Decision making after reviewing the report
- Legal background
  - NSSC Administrative order (2011. 5.13)
  - Incident Response Manual for Nuclear Facilities (Rev. 1, 2021.6)
  - KINS Procedure "Procedure for the Incidents/Accidents investigation and information release"

#### Corrective measures

- □ Items for the corrective measures
  - KINS: Review the licensee's corrective measures whether they are chosen properly based on the licensee's investigation.
- Tracking on the corrective measures of reported events
  - NSSC, "Regulations on the Technical Standards for Nuclear Reactor Facilities, Etc." Article 58 (Reflection of Operating Experience)
  - KINS performs periodic inspection on the status of the corrective measures derived after the events.

# III.3 Rating of the Event Scale



#### User's Manual 2008 Edition







# INES

#### THE INTERNATIONAL NUCLEAR AND RADIOLOGICAL EVENT SCALE

he INES Scale is a worldwide tool for communicating to the public in a consistent way the safety significance of nuclear and radiological events.

Just like information on earthquakes or temperature would be difficult to understand without the Richter or Celsius scales, the INES Scale explains the significance of events from a range of activities, including industrial and medical use of radiation sources, operations at nuclear facilities and transport of radioactive material.

Events are classified on the scale at seven levels: Levels 1–3 are called "incidents" and Levels 4–7 "accidents". The scale is designed so that the severity of an event is about ten times greater for each increase in level on the scale. Events without safety significance are called "deviations" and are classified Below Scale / Level 0.

Major Accident Level 7

Serious Accident Level 6

Accident with Wider Consequences Level 5

Accident with Local
Consequences
Level 4

Serious Incident Level 3

> Incident Level 2

Anomaly Level 1

NO SAFETY SIGNIFICANCE (Below Scale/ Level 0) **INES** classifies nuclear and radiological accidents and incidents by considering three areas of impact:

**People and the Environment** considers the radiation doses to people close to the location of the event and the widespread, unplanned release of radioactive material from an installation.

Radiological Barriers and Control covers events without any direct impact on people or the environment and only applies inside major facilities. It covers unplanned high radiation levels and spread of significant quantities of radioactive materials confined within the installation.

**Defence-in-Depth** also covers events without any direct impact on people or the environment, but for which the range of measures put in place to prevent accidents did not function as intended.

#### **EXAMPLES OF EVENTS AT NUCLEAR FACILITIES**

|   | People and Environment   | Radiological Barriers<br>and Control  | Defence-in-Depth   |
|---|--|---|--|
| 7 | Chernobyl, 1986 — Widespread health and environmental effects. External release of a significant fraction of reactor core inventory. |   |  |
| 6 | Kyshtym, Russia, 1957 — Significant release of radioactive material to the environment from explosion of a high activity waste tank. |   |  |
| 5 | Windscale Pile, UK, 1957 — Release of radioactive material to the environment following a fire in a reactor core.                    | Three Mile Island, USA, 1979 —<br>Severe damage to the reactor core.  |  |
| 4 | Tokaimura, Japan, 1999 — Fatal overexposures of workers following a criticality event at a nuclear facility.                         | Saint Laurent des Eaux, France,<br>1980 — Melting of one channel of<br>fuel in the reactor with no release<br>outside the site. |  |
| 3 | No example available   | Sellafield, UK, 2005 — Release of large quantity of radioactive material, contained within the installation.                    | Vandellos, Spain, 1989 — Near accident caused by fire resulting in loss of safety systems at the nuclear power station.                  |
| 2 | Atucha, Argentina, 2005 — Overexposure of a worker at a power reactor exceeding the annual limit.                                    | Cadarache, France, 1993 — Spread of contamination to an area not expected by design.  | Forsmark, Sweden, 2006 — Degraded safety functions for common cause failure in the emergency power supply system at nuclear power plant. |
| 1 |  |   | Breach of operating limits at a nuclear facility.  |

## **Event Rating System**

#### ☐ History

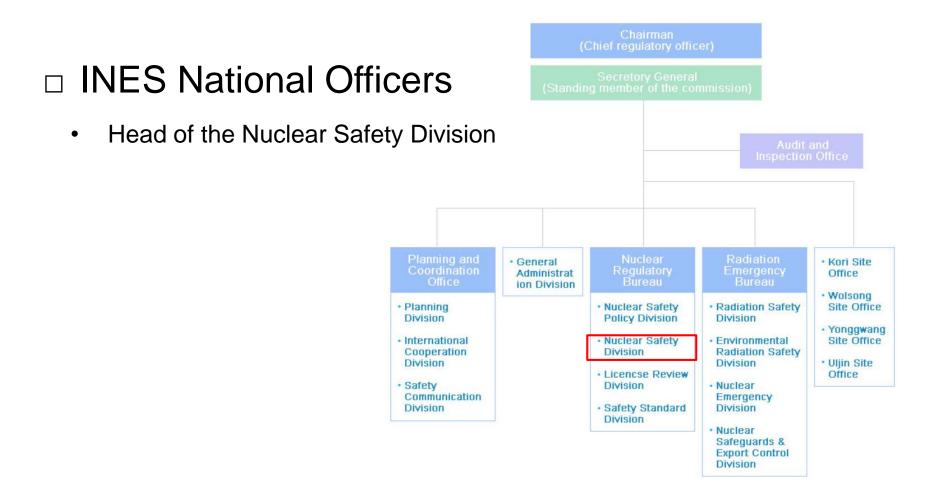
- Introduced IAEA INES in March, 1993
- Nuclear Event Evaluation Committee (NEEC)
  - held 111 times quarterly, and rated 430 events since its 1st meeting in 1993

## Event Rating System (cont.)

#### Event Scale Evaluation Committee

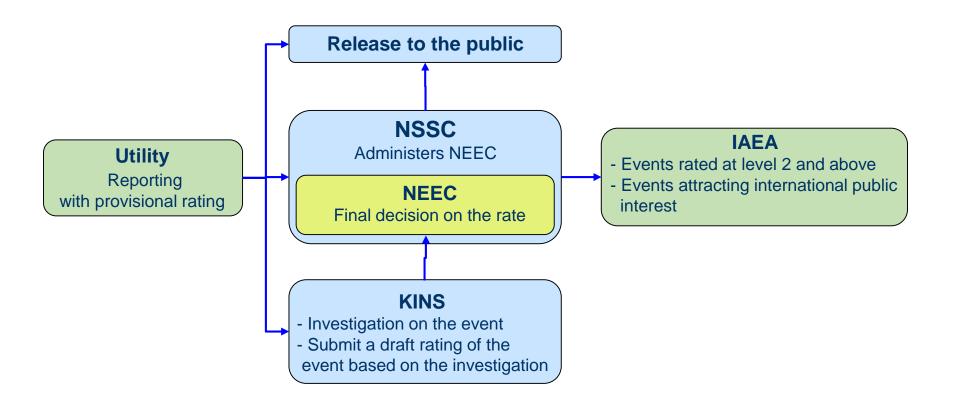
- The NSSC(Nuclear Safety and Security Commission) was established as a government regulatory authority(Oct 26, 2011).
- Event evaluation committees have been administered by NSSC since March 2012, according to NSSC Notice 2012-85 (Currently 2018-03).
- Members of Evaluation Committees consisted of government officials and nuclear/radiation field experts including some KINS staffs with 2year term of service.

## Event Rating System (cont.)



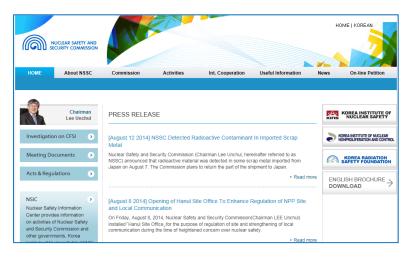
### **Event Rating Process**

■ Institutions and Work Arrangements

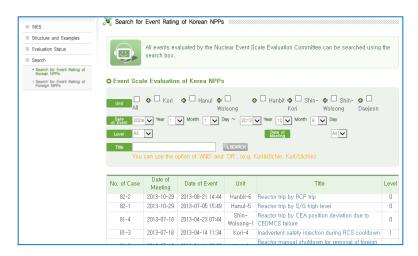


#### Information Release to the Public

■ Web sites for the release of information



NSSC web site (<a href="http://www.nssc.go.kr">http://www.nssc.go.kr</a>)



OPIS (<u>http://opis.kins.re.kr</u>): <u>Operational</u>
 <u>Performance Information System for NPP</u>

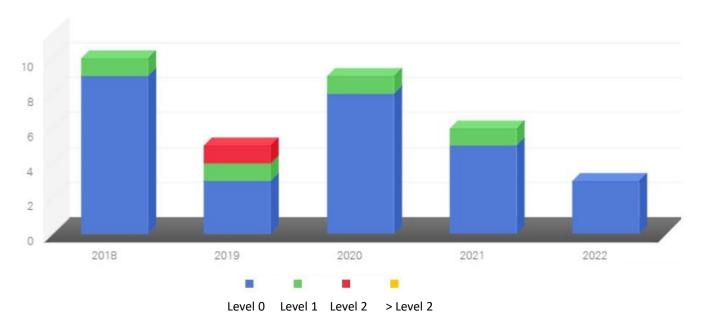
## Recent Events Rating in Korea

□ Totally 430 events rated by NEEC since 1993~2022.8

Level 2: 4 events

Level 1 : 30 events

Level 0 : 396 events



#### Summary

- □ NSSC Notice No. 2020-3 defines.
  - Types of events to report, timings and methods of the reporting
  - Rating of the reported events, release of the information
  - Investigation of the events, management of the information

# IV. The National OEF Program in Korea



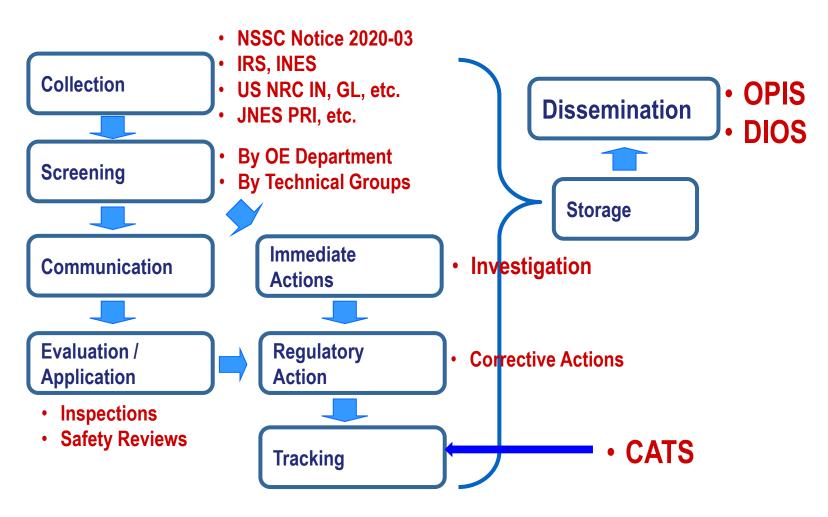
### VI.1 National OEF(NOEF) program

- Regulatory Body (NSSC and KINS)
  - Perform investigation on the reported events, issuance of the corrective actions, and information release to the public according to NSSC Notice "2020-03"
  - Collection, categorization, storage and dissemination of the OE information
  - Screening and implementation of safety issues
  - Utilization of regulatory experiences obtained through inspection and review activities

#### NPP Utility (KHNP)

- Storage and dissemination of important domestic and foreign information (KONIS)
- Accumulation of information on incident, maintenance and routine works including surveillance test (DREAMS)

#### [Overall OEF Process]



- Reference: IAEA Safety Guide No. NS-G-2.11 "A System for the Feedback of Experience from Events in Nuclear Installations

#### Backgrounds of the NOEF

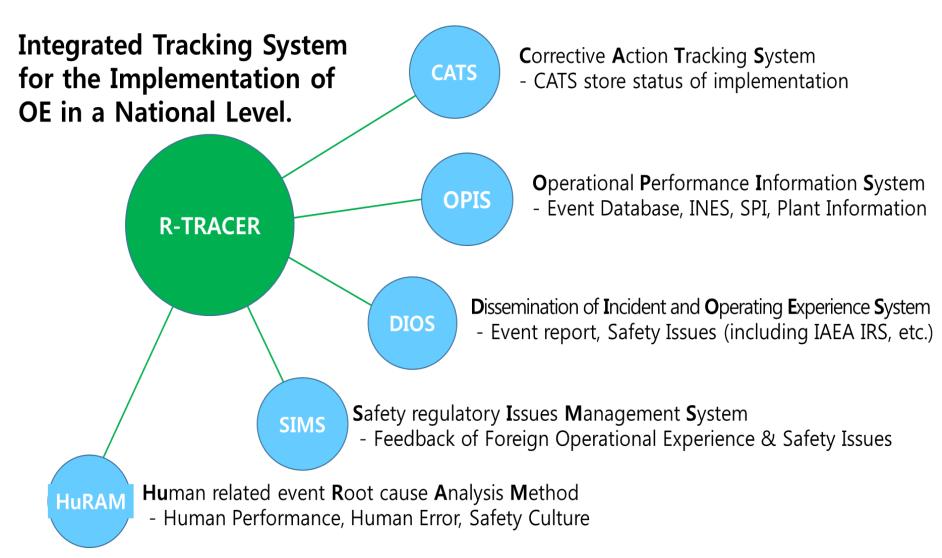
- National concerns on the nuclear installations including NPPs
- The Public, NGOs and Stakeholders want to know the information about incidents and failures of equipment in NPPs.
- IT-based tools can fulfill these demands, and promote the public confidence in the nuclear safety.
- This also implements the functions of OEF by Disseminating and Storing the related information.

#### OEF Environments

- Huge amount of OEF data
- → thus, mostly based on the computer and DB system.
- → Promote and facilitate OE activities.
- Effective and efficient implementation of OEF requires development of web-based OEF Tools
  - Web-based OEF Tools can reduce the time to collect/screen/evaluate/analyze and investigate/disseminate the OEF data, and
  - provide effective ways to satisfy both domestic and international guidelines of the OEF.

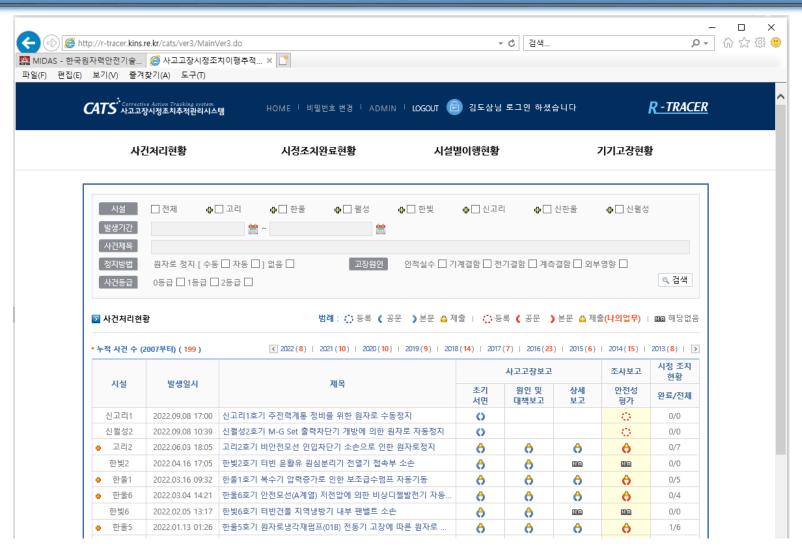
#### **IV.2 Tools for the national OEF**

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# CATS Corrective Actions Tracking System (1)



## CATS Corrective Actions Tracking System (2)

- Web-based Interactive system operated by regulatory body and Utility.
- Developed, especially, to trace and the Utility's implementation of the corrective actions
- CATS stores all the event related information
  - KHNP: Basic information, Preliminary and detailed reports,
     Planning and Implementation of CA
  - KINS: Investigation Report, Recommendation for the CA, Evaluation Report for the CA
  - NSSC: Requesting implementation of CA, Approval for the Restart, Approval of CA

## CATS Corrective Actions Tracking System (3)

#### **Implementation status**



📤 한국원자력안전기술원

신월성1발

한수원 본사

신월성2호기

5차(2022)

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2022.06.18 ~ 2022.08.12 2019년 01월 ~ 2022년 06월



▶ 등록

▶ 등록

# DIOS Dissemination of Incident and OE System (1)

HOME | ADMIN | LOGOUT



DiOS 운전경험정보관리시스템



🕞 김도삼님 로그인 하셨습니다

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QUICK MENU

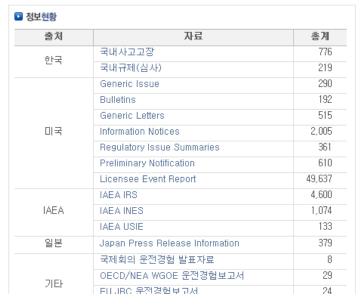
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운전경험정보 검색

**111** 정보현황

9

분류현황







## DIOS Dissemination of Incident and OE System (2)

- DIOS is an IT based archival system for the OEF system
  - manages huge amounts of OE data
  - efficiently handles the data by utilizing web-based technique
  - data is stored with categorized codes
- Developed for the efficient storage, management and dissemination of Incidents and OE related data and Safety Issues
  - Events, Regulatory issues raised from the safety review and inspection
  - Events and Safety issues from IAEA, USNRC, NRA, etc.,

# OPIS Operational Performance Information System





## OPIS Operational Performance Information System

- Major contents of OPIS
  - Nuclear Event Evaluation Database (NEED)
  - INES (International Nuclear Event Scale)
  - SPI (Safety Performance Indicator)
  - Status of World NPP
  - English Version of the OPIS

## OPIS Operational Performance Information System

#### Annual OEF workshop since 2003

- To exchange information on events from foreign and domestic nuclear facilities
- To discuss the event-related regulation perspectives
- To improve the national OEF program

#### International Cooperation Activities

- Sharing and discussion on lessons learned from OE among the countries
- Participation on international OEF-related workshop

# Always we keep watching our Atomic Power



